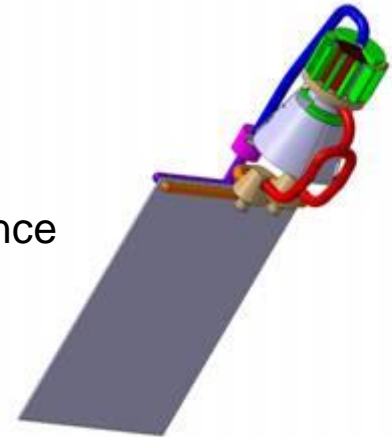


# Coupled 1D thermohydraulic – 3D thermomechanical study of a space nuclear reactor



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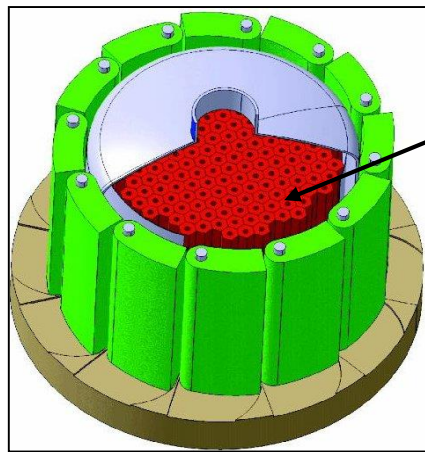
# The OPUS System

## To maintain a waking state activity

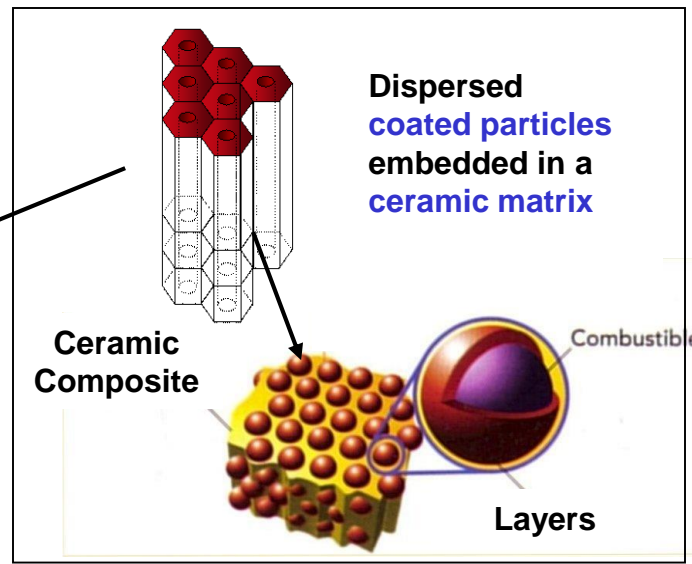
Pre-conceptual design studies of what would be a NEP system in the range 100-500 kW<sup>e</sup>  
 ⇒ **OPUS studies**

## Preliminary basic options

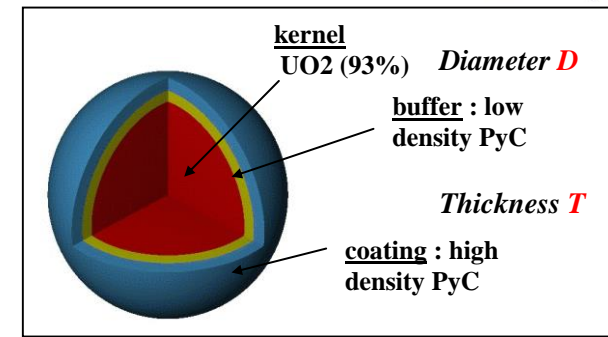
- Gas-cooled reactor
- Fast neutron spectrum
- Dynamic conversion cycle



Reactor Core



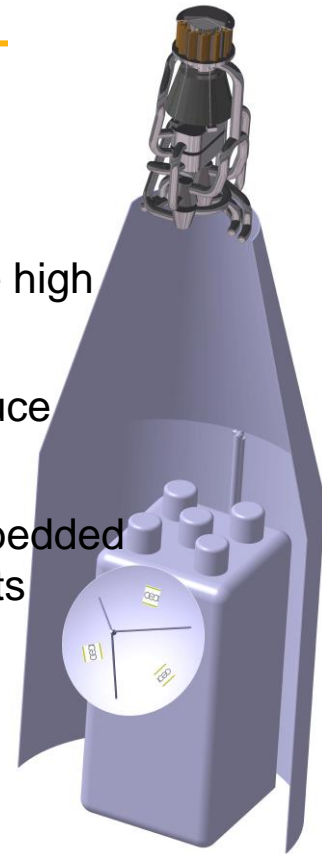
Nuclear fuel elements



Particle design

## Basic core design

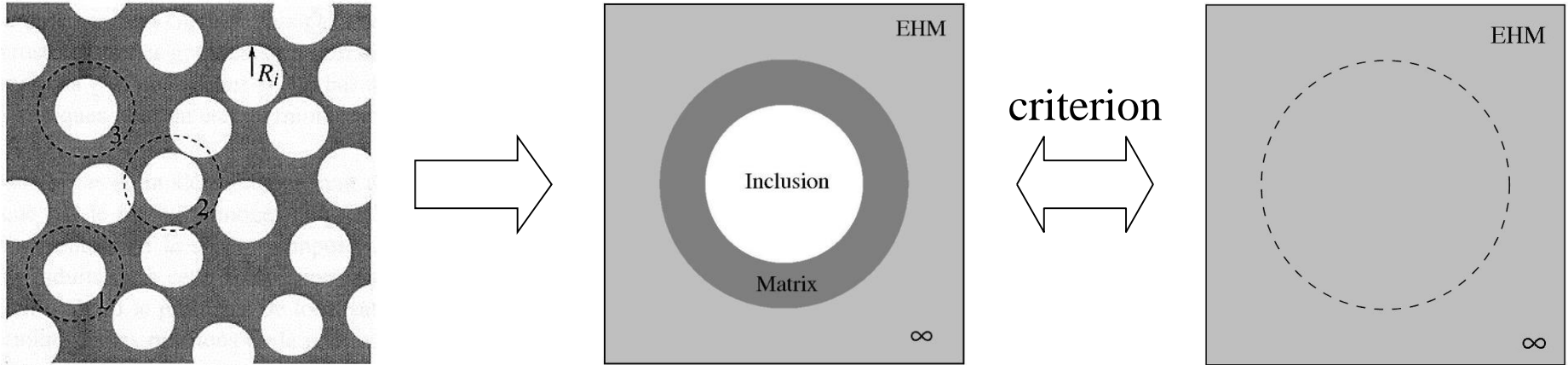
- **Refractory material** ⇒ to achieve high temperature (1300 K at the outlet)
- **Fast neutron spectrum** ⇒ to reduce the critical mass
- **Coated fuel particles (BISO)** embedded in hexagonal graphite fuel elements



# Composite fuel modeling\*

- Material heterogeneity simplification

- Equivalent Homogeneous Media



- Effective thermoelastic properties :

$\lambda^*$ ,  $E^*$ ,  $\nu^*$ ,  $\alpha^*$  functions of  $\{\lambda_i, E_i, \nu_i, \alpha_i\}_{1 \leq i \leq n}$  and phase proportions  $\{c_i\}_{1 \leq i \leq n}$

- Local strains and stresses :

$$\underline{\underline{\varepsilon}}_i = \mathcal{A}_i : (\underline{\underline{E}} - \underline{\underline{E}}^{th} - \underline{\underline{E}}^{ir}) + \sum_i \mathcal{P}_{ij} : (\underline{\underline{\varepsilon}}_j^{th} + \underline{\underline{\varepsilon}}_j^{ir}) \quad \text{avec : } \underline{\underline{E}}^{ir} = \sum_i \chi_i \mathcal{B}_i : \underline{\underline{\varepsilon}}_i^{ir}$$

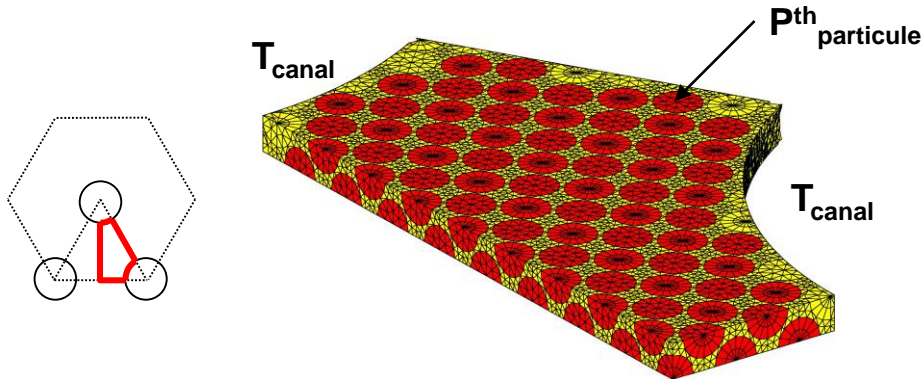
$$\underline{\underline{\sigma}}_i = \mathcal{C}_i : (\underline{\underline{\varepsilon}}_i - \underline{\underline{\varepsilon}}_i^{th} - \underline{\underline{\varepsilon}}_i^{ir})$$

(\*) E. Hervé, A. Zaoui, “N-layered inclusion-based micromechanical modelling”, Int. J. Engng Sci., **31** (1993) 1-10.

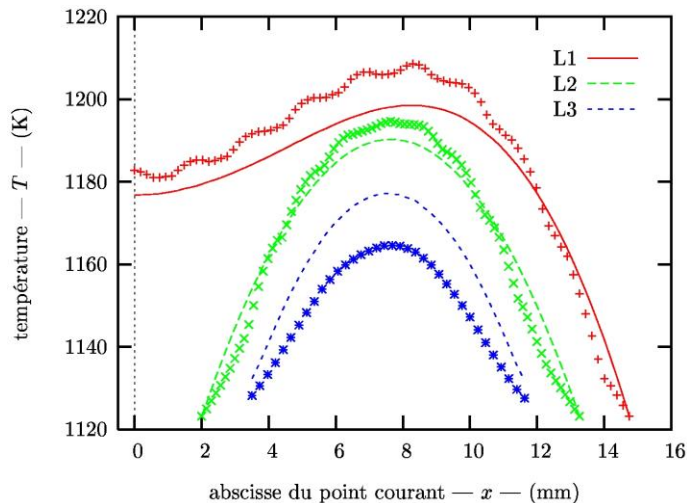
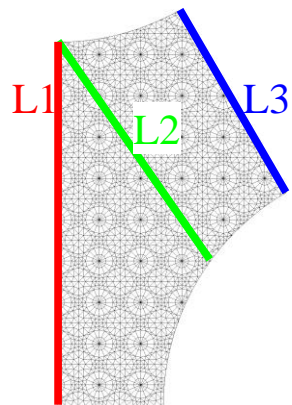
# Composite fuel modeling: validation & ongoing studies

## Fuel Element cross-section modelling

- Thermomechanical modelling



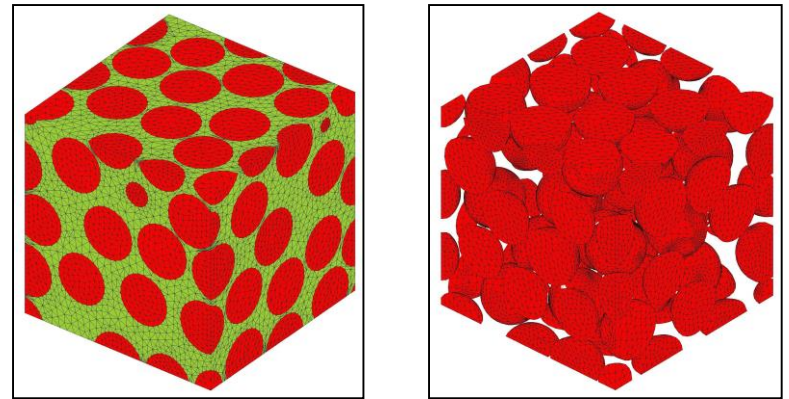
- Results



## Heterogeneous media modelling

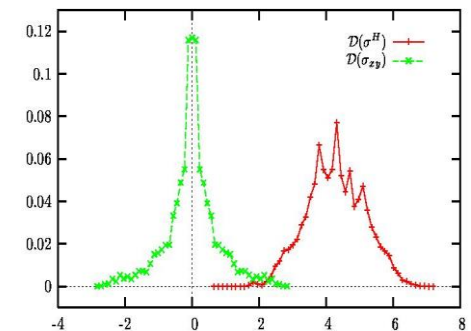
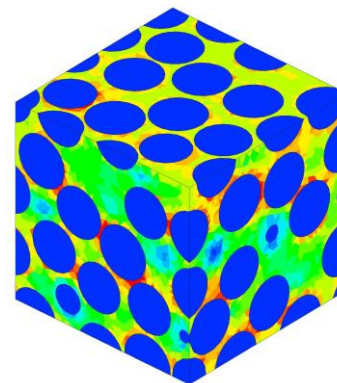
(F. Di Paola PhD Thesis)

- Mesh generation



Particle Volume Fraction ~ 43%

- Mechanical Characterisation

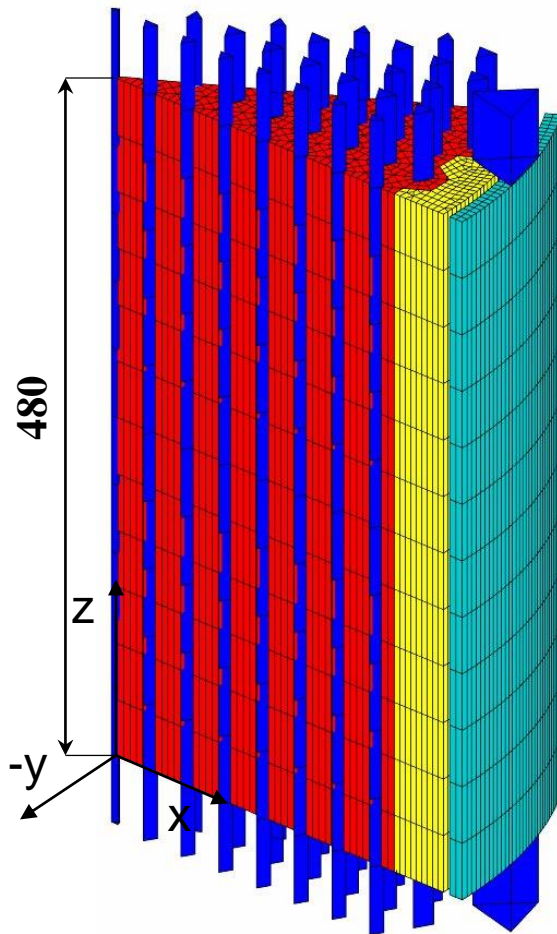


Local stress and strain fields



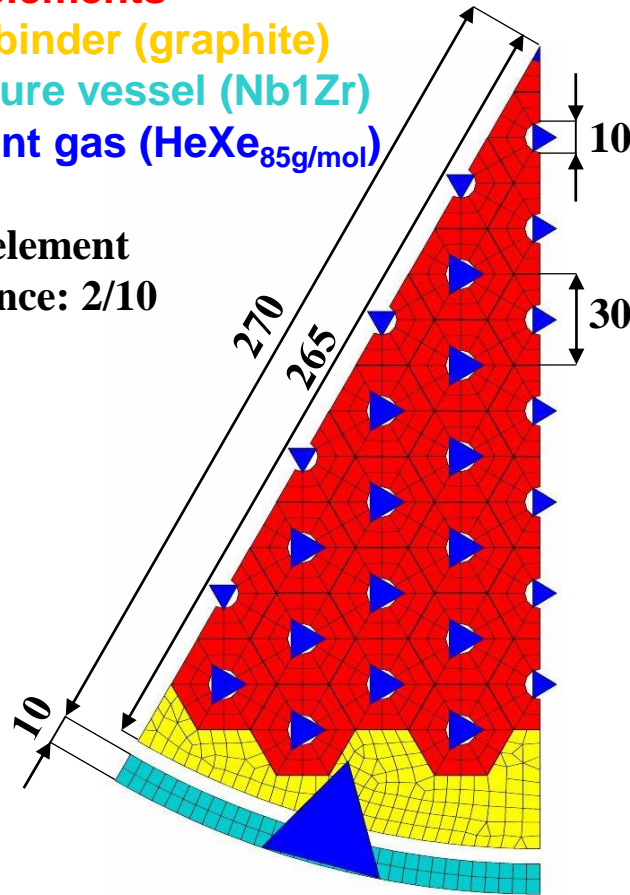
# Geometry / Finite Element Mesh

1/12<sup>th</sup> of the Reactor

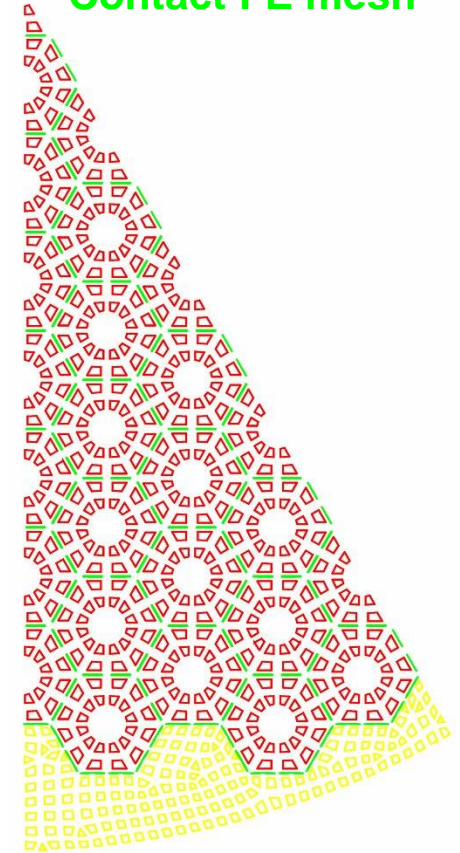


**Fuel elements**  
**Core binder (graphite)**  
**Pressure vessel (Nb1Zr)**  
**Coolant gas (HeXe<sub>85g/mol</sub>)**

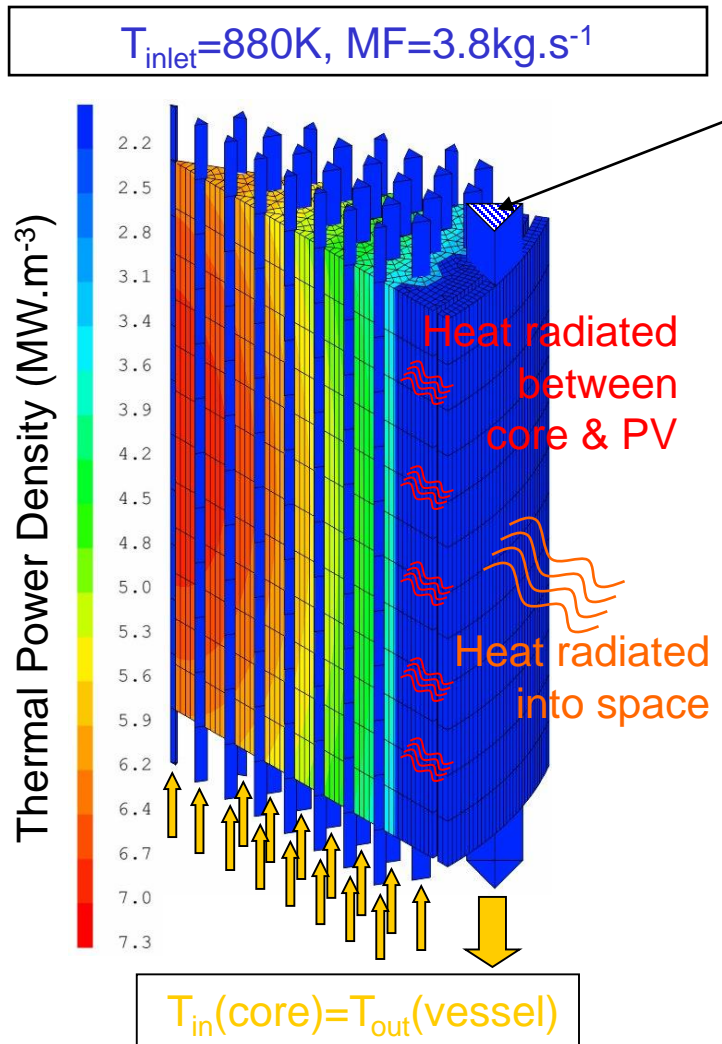
**Inter-element  
clearance: 2/10**



**Contact FE mesh**



# 1D-thermohydraulic / 3D-thermal coupling



## Thermohydraulic:

- **1D modelling:** Mass conservation & Energy balance, but **pressure drop neglected**
- **He-Xe proportion parameterized**
- **State law:** **ideal gas law**
- **Boundary conditions**  
 $T_{inlet} = 880K$   
 $MF_{inlet} = 3.8kg/s \Rightarrow T_{outlet} = 1300K$
- **Convective heat exchange:** **Colburn** or **Dittus-Bolter**  
 $Nu = 0.023Re^{0.8}Pr^{(1/3 \text{ or } 0.4)} \Rightarrow h = Nu \lambda_{He}(T_m)/d$

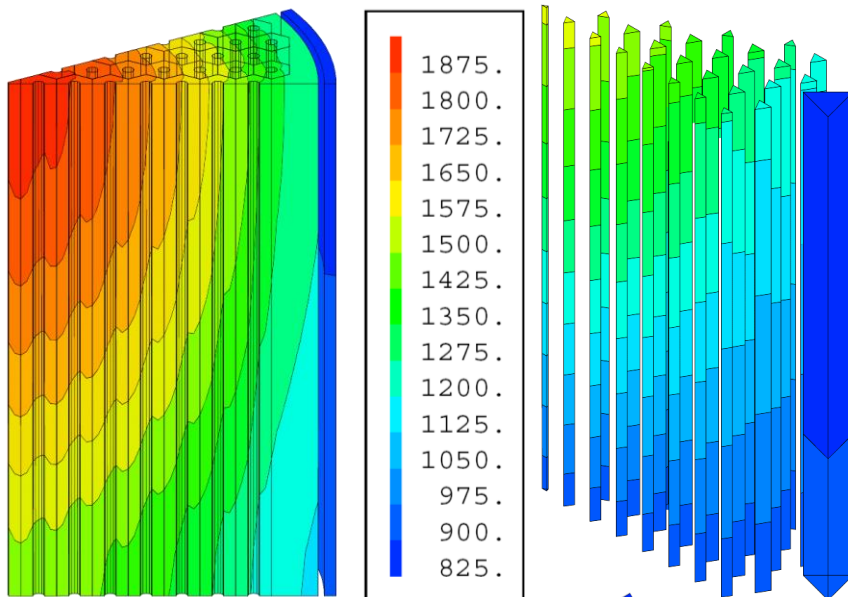
## Thermal:

- **3D modelling:** **core = continuous media**
- **Thermal loading:** Power density fit to **TRIPOLI results**  

$$p(r,z) = p_{max} J_0(\alpha r) \cos(\beta \tilde{z}), 10kW$$
- **Boundary conditions:**
  - **Convective heat exchange** with the coolant gas on: Channel / outer Core / inner Pressure Vessel surfaces
  - **Radiant heat exchange** between: outer Core / inner PV surfaces & outer PV surface

# Temperature

$$T_{\text{outlet}} = 1300\text{K}$$

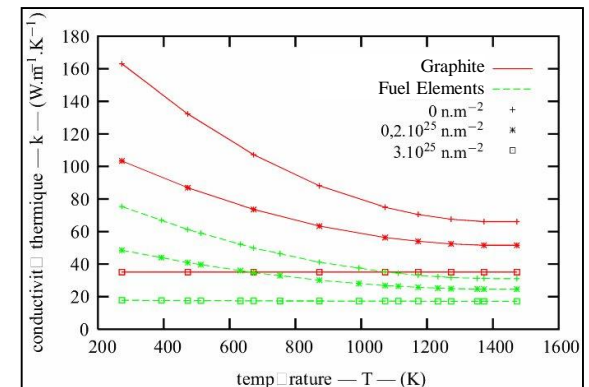


## Results & Evolution from 0 to 2000 EFPD

Structures		
	0 JEPP	2 000 JEPP
Température maximale	1 807	1 870
Extrema en sortie du cœur (centre > périphérie) et écart / gradient radial	1 807 / 1 203 604 / 2 279 $\text{K.m}^{-1}$	1 870 / 1 137 735 / 2 774 $\text{K.m}^{-1}$
Extrema assemblage le plus chaud et écart / gradient axial	1 807 > 1 427 380 / 792 $\text{K.m}^{-1}$	1 870 > 1 417 453 / 944 $\text{K.m}^{-1}$
Extrema baffle	1 302 > 1 130	1 280 > 1 076
Extrema cuve	946 > 906	926 > 894
Gaz caloporteur		
Température maximale	1 474	1 491
Température moyenne en sortie du cœur	1 301,6	1 302,1
Extrema en sortie du cœur (centre > périphérie) et écart entre le canal le plus chaud et le plus froid	1 474 / 1 153 321	1 491 / 1 124 367
Extrema dans le canal le plus chaud et écart / gradient axial	1 474 > 953 521 / 1 085 $\text{K.m}^{-1}$	1 491 > 936 555 / 1 156 $\text{K.m}^{-1}$

Due to the decrease of thermal conductivity under irradiation :

*Thermal conductivity  
Vs. Temperature  
& Irradiation Dose*

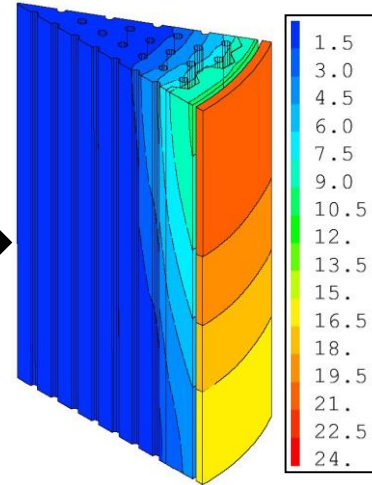


Temperature (K) Structures & Coolant gas

# Thermohydraulic-Thermal Simulations : Summary

## Main results

- $T_{\max}$  : 1909K  $\rightarrow$  **2001K ( $\sim 1730^{\circ}\text{C}$ ) !**
- **Max. Temp.**  $\nearrow$  under irradiation due to  $\searrow$  Graphite **Thermal Conductivity**
- **Radiant Heat Exchange**  $\searrow$  PV Temperature Gradients ( $\Delta T \searrow \sim 20\text{K}$ )  $\rightarrow$
- **Convective Heat Exchange Analogy** not fully satisfying:  
Experimental data on **He-Xe coolant** in such conditions ?



## Modelling improvements

- **Thermal modelling**: reactor core modelled as a continuous media but **temperature jumps at the fuel element interface** should be very important...
- **Thermohydraulic BC**: mass flow prescribed at the core inlet but **coolant flow** should not uniformly distributed over the channels  
 $\Rightarrow$  thermal gradients could be **even more important** in the core

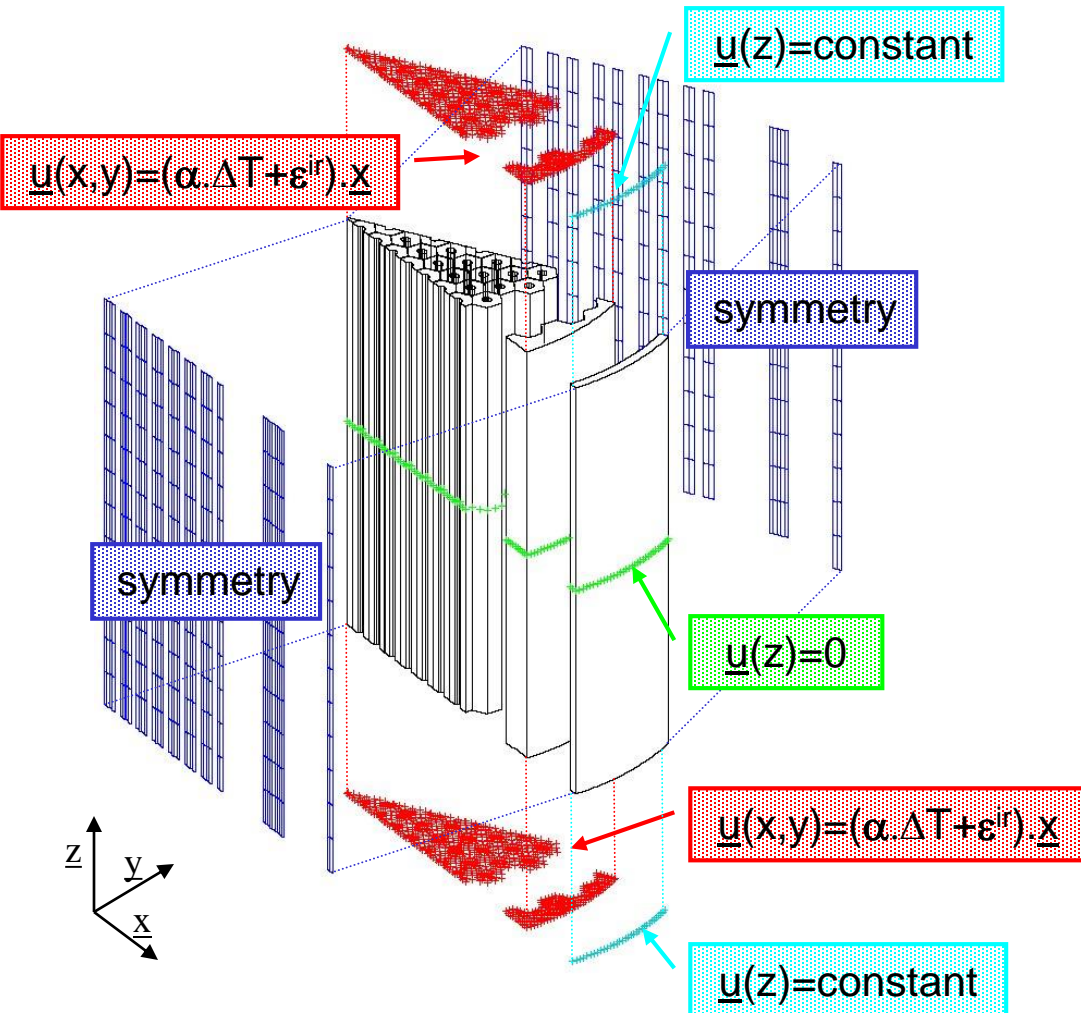
## Remark on the Simulation Tool

- **Temperature field** of the reactor **fully determined** in 3D by setting **only 3 parameters**:  
 $\Rightarrow$  Inlet Coolant Temp. / Inlet Mass Flow / Thermal Power of the Core



# Mechanics: Boundary Conditions and Loadings

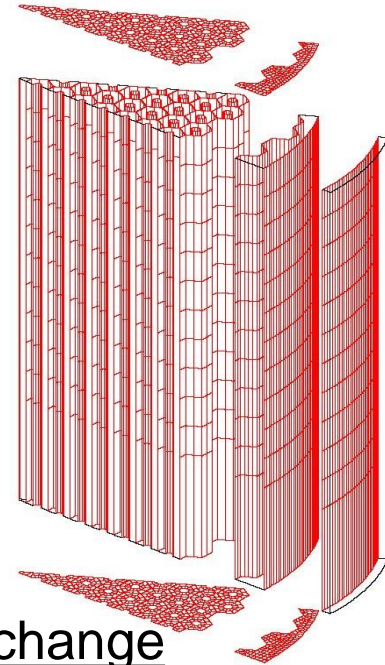
## Cinematic B.C.



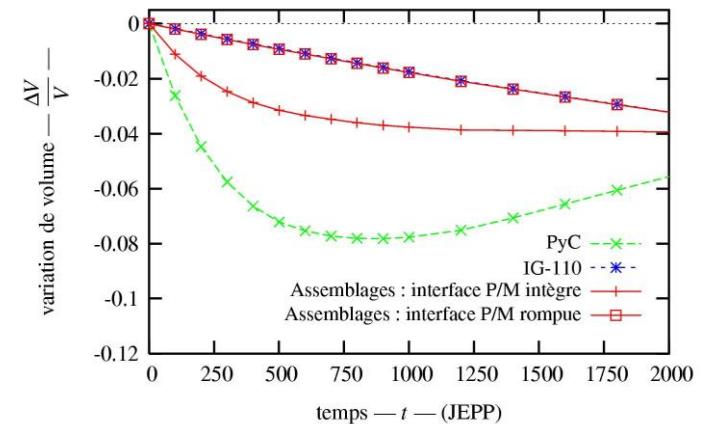
## Static B.C.

Pressure loading applied on the red surfaces :

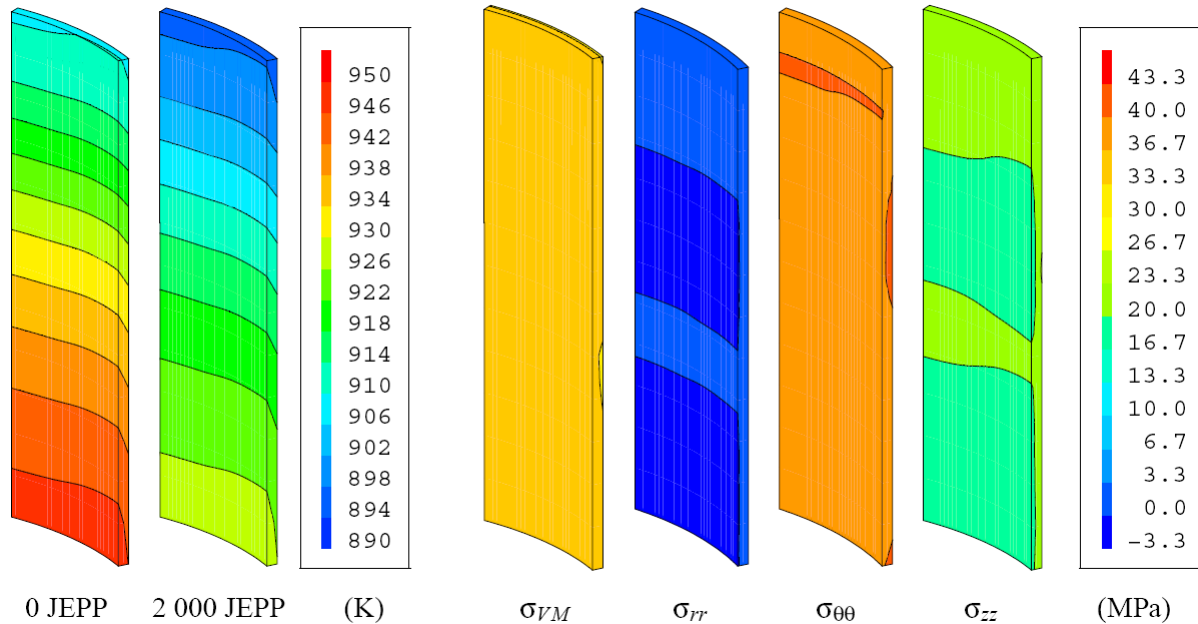
$$P = 1,4 \text{ MPa}$$



## Irradiation vol. change



# Mechanics: Pressure Vessel



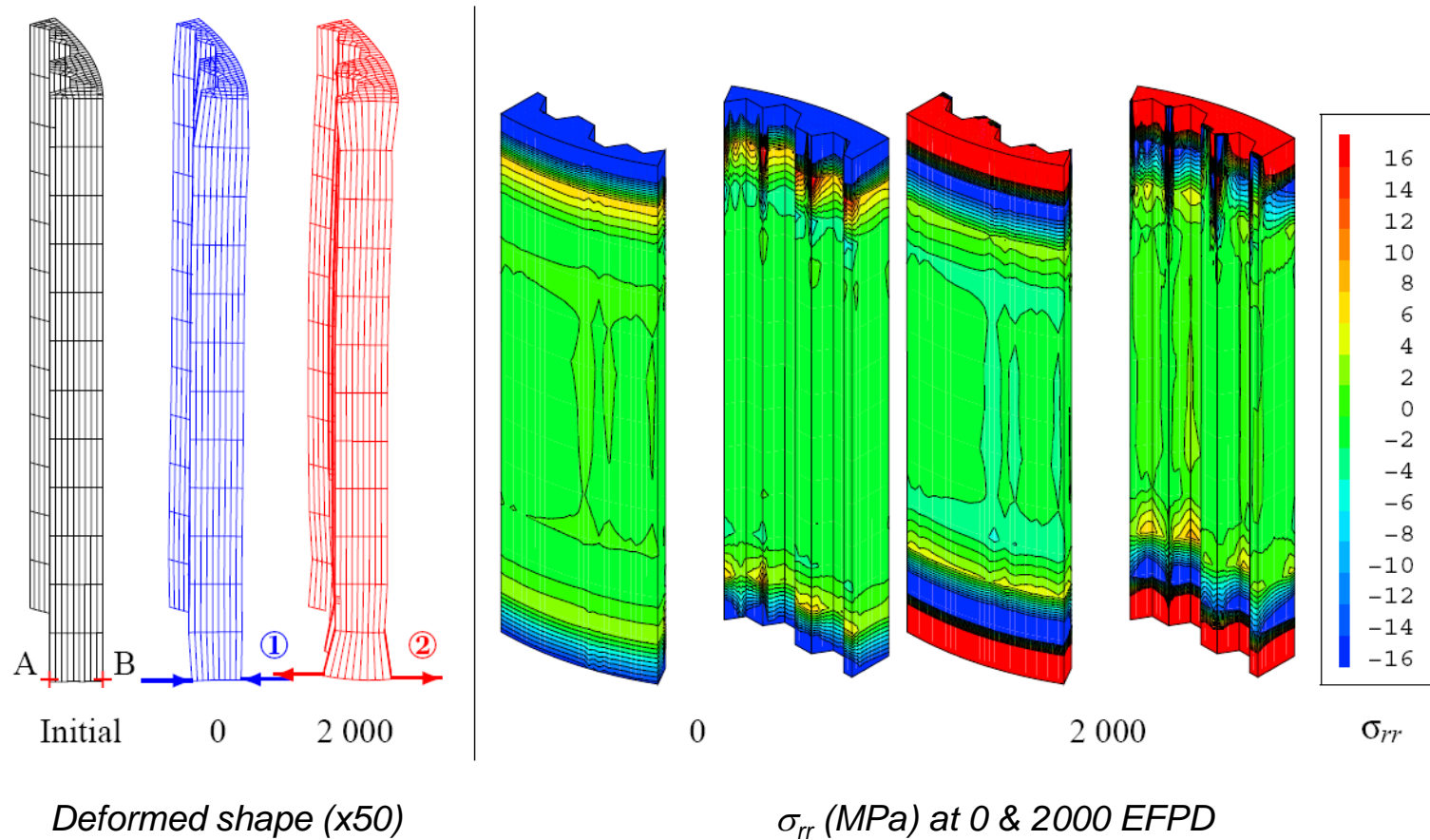
## Summary

- Stress state
  - Max ( $\sigma_{\theta\theta}$ )  $\sim$  41MPa
  - $\sim$  P.R<sub>m</sub>/e (38.5MPa)
  - Max ( $\sigma_{zz}$ )  $\sim$  23MPa
  - $\sim$  P.R<sub>m</sub>/2e (19.5MPa)
  - $\Rightarrow$  thermal stresses very low
- Mechanical Design
  - Static :  $\sigma^y \sim$  110 MPa  $\sim$   $\sigma^{\max} / 3$
  - $\Rightarrow$  thickness = 10 mm  $\sim$  ok
  - But : M(core)  $\sim$  700kg
  - $\Rightarrow$  PV design = dynamic loadings (lift-off  $\sim$  4g)

Temps (JEPP)	$\sigma_{VM}$ (MPa)	$\sigma_{\theta\theta}$ (MPa)	$\sigma_{zz}$ (MPa)	Autres composantes (MPa)
0	$31 < \sigma_{VM} < 35$	$34,5 < \sigma_{\theta\theta} < 41,5$	$16 < \sigma_{zz} < 23$	$ \sigma  < 6$
2 000	$32 < \sigma_{VM} < 35$	$34,5 < \sigma_{\theta\theta} < 41,0$	$16 < \sigma_{zz} < 22$	$ \sigma  < 6$

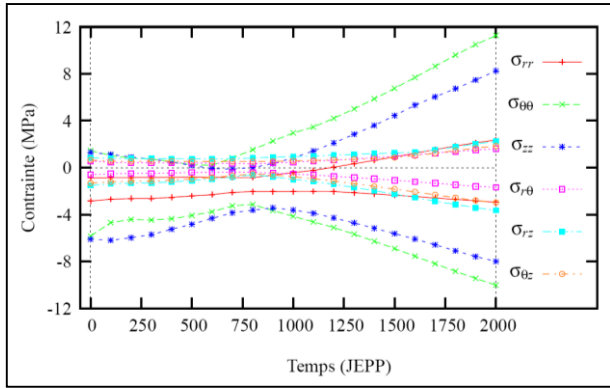
# Mechanics: Core Binder

with Top & Bottom Cinematic B.C.

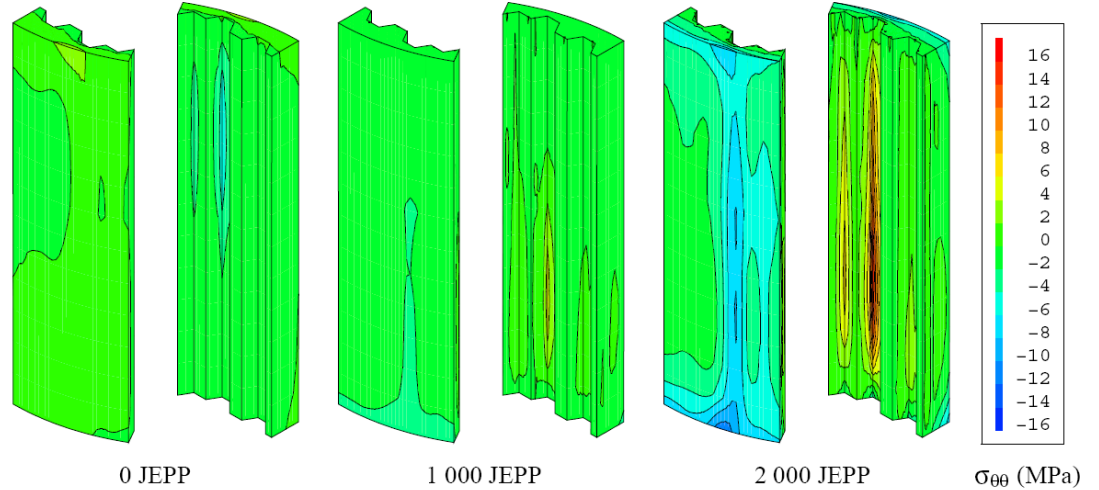


# Mechanics: Core Binder

without Top & Bottom Cinematic B.C.



Stresses (MPa) Vs. Time (EFPD)

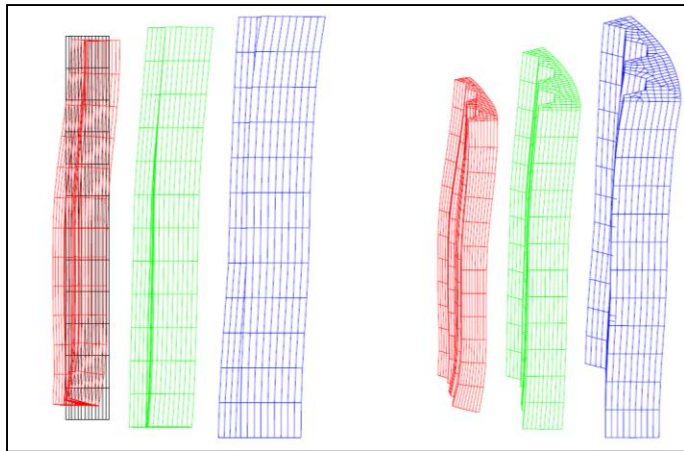


0 JEPP

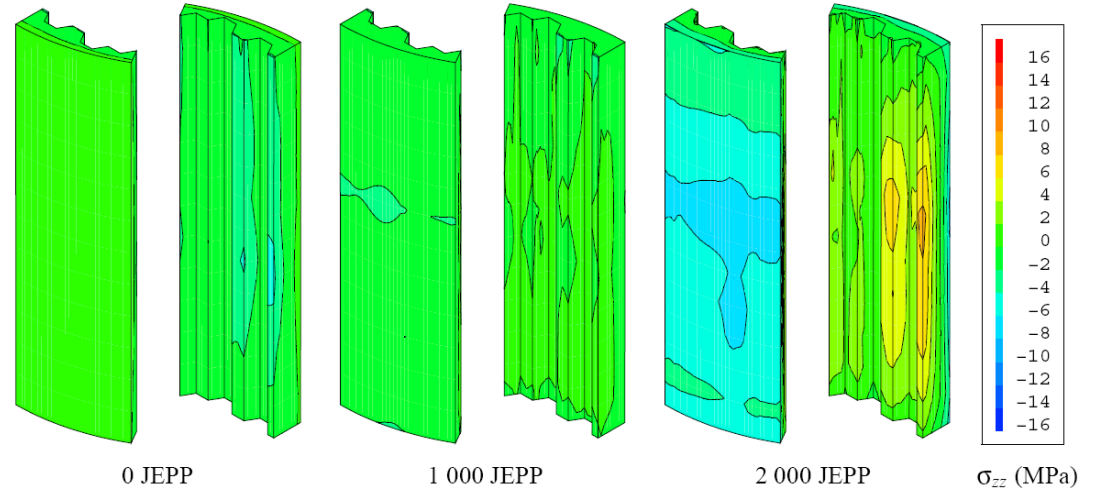
1 000 JEPP

2 000 JEPP

$\sigma_{\theta\theta}$  (MPa)



Deformed Shape : (x20) axially,  
(x200) in the  $(O, r, \theta)$  plane



0 JEPP

1 000 JEPP

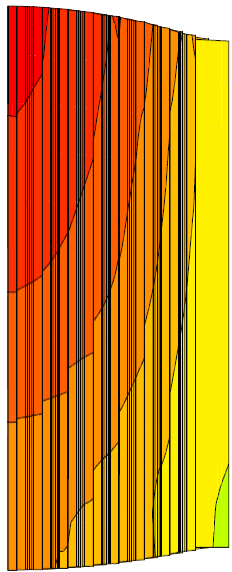
2 000 JEPP

$\sigma_{zz}$  (MPa)

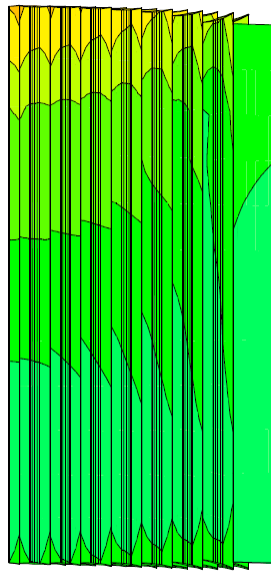


# Mechanics: Reactor Core

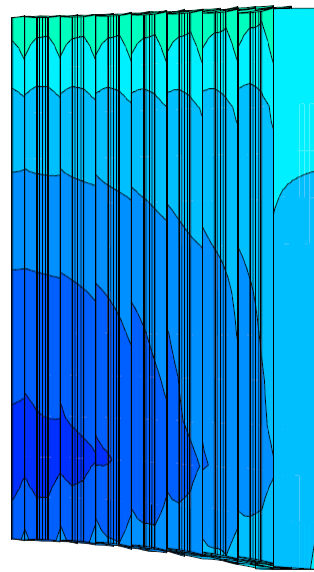
## Deformed Shape (x50)



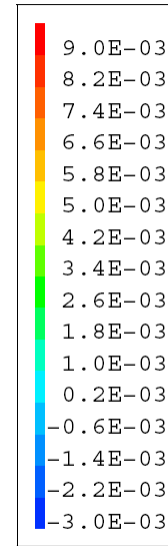
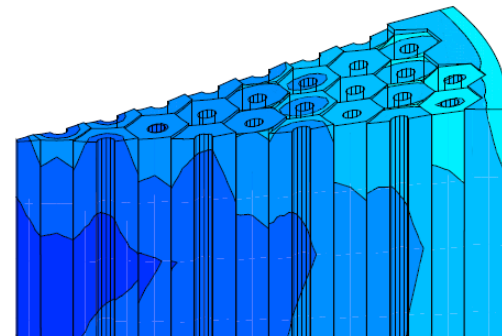
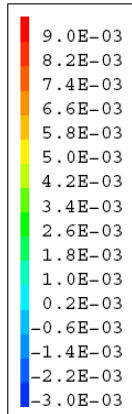
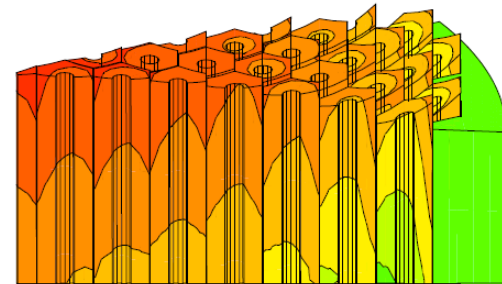
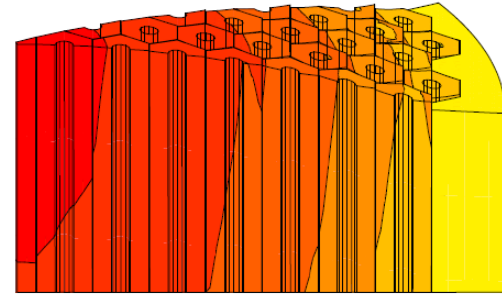
0 JEPP



1 000 JEPP



2 000 JEPP

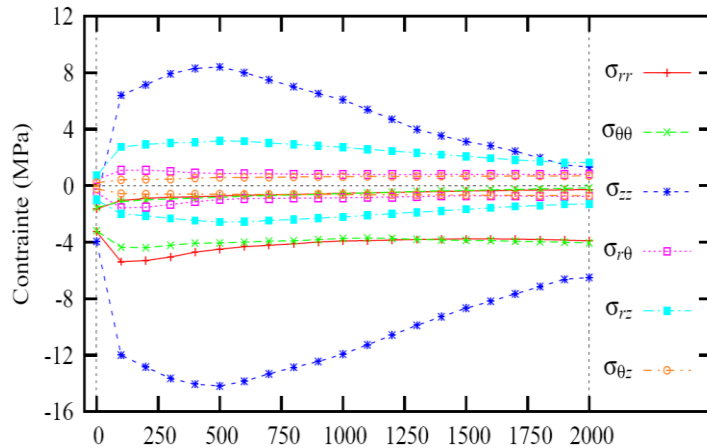
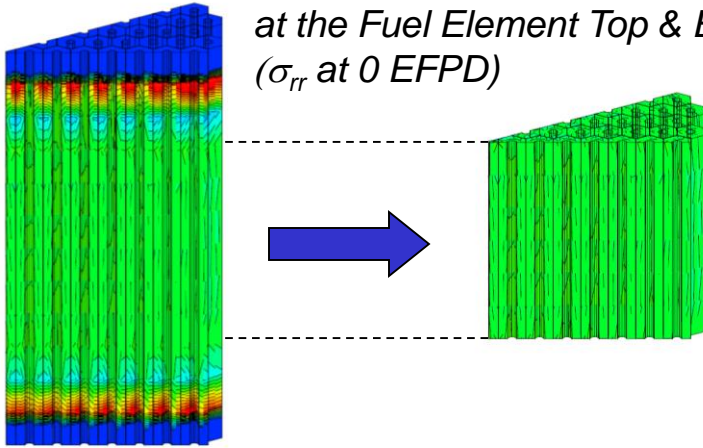
 $\epsilon_{zz}$ 

	0 JEPP	2 000 JEPP
Partie inférieure ( $z = 0$ )	$\Delta u_z = 0,56$ mm	$\Delta u_z = -0,52$ mm
Partie supérieure ( $z = H1$ )	$\Delta u_z = 0,83$ mm	$\Delta u_z = -0,2$ mm

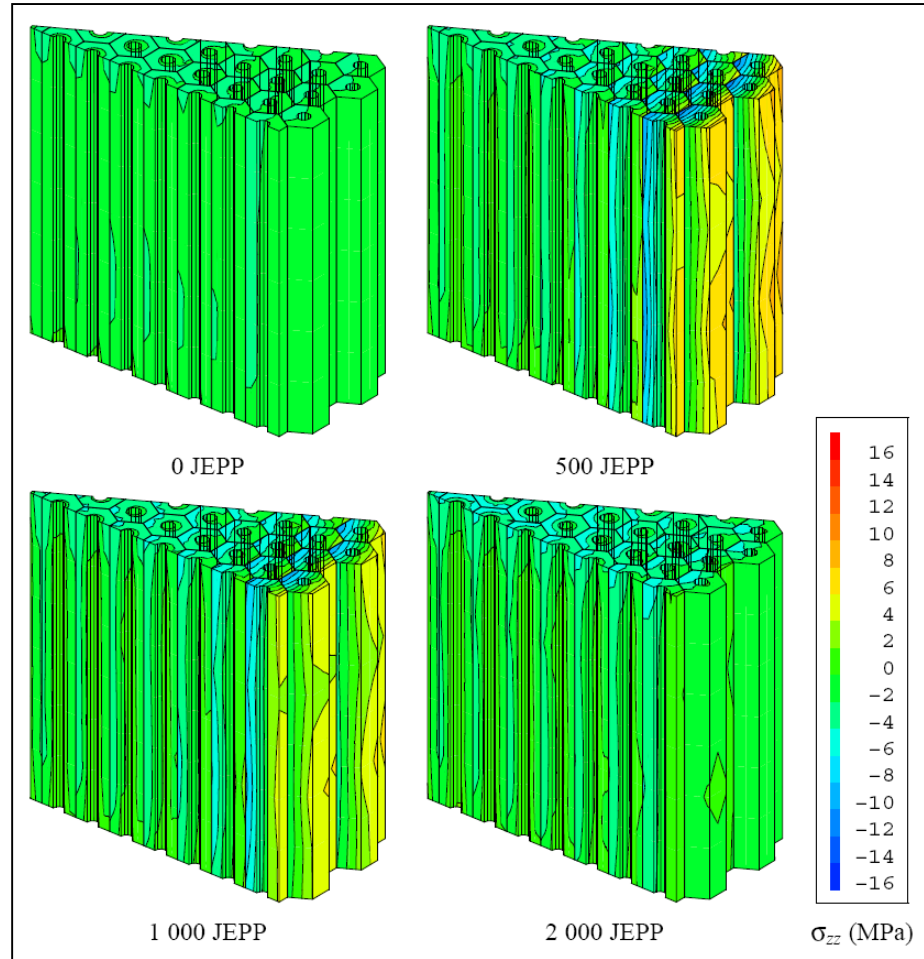
*Difference of Axial Displacements (Core Top & Bottom)*

# Mechanics: Fuel Elements

Problem with the Cinematic B.C.  
at the Fuel Element Top & Bottom  
( $\sigma_{rr}$  at 0 EFPD)



Stresses (MPa) Vs. Time (EFPD)



Effective stress  $\sigma_{\theta\theta}$  in the fuel element graphite matrix

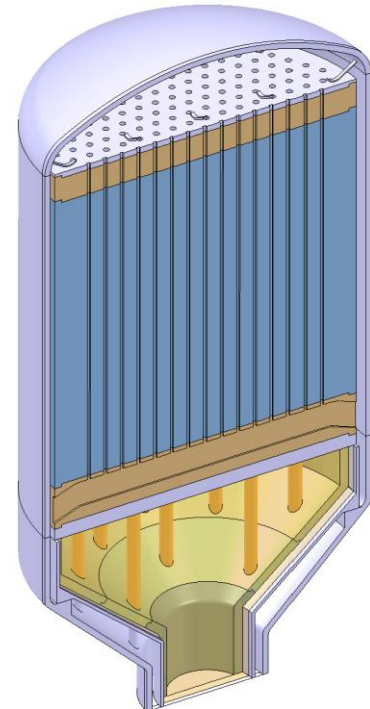
# Space Reactor Design: Main Conclusions

## Thermal-Thermohydraulic

- $T_{\max}$  : 1909K → 2001K (~1730°C)
- Temperature jumps and coolant flow distribution ⇒  $T_{\max} \sim 2300$  K (~ 2000°C)  
⇒ To optimize channel diameter to homogenize core temperature
- Temperature field fully determined over the reactor only by setting :  
⇒ Inlet Coolant Temp. / Inlet Mass Flow / Thermal Power of the Core

## Mechanics

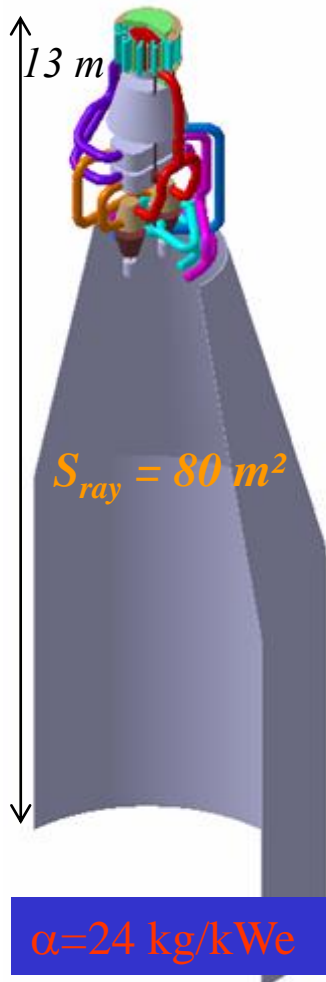
- PV: Current design satisfied nominal static loading but:  
Mechanical design should more depend on dynamic loadings (lift-off)
- Core Binder:  $\Delta u$  (BC) ~ 1/10mm ⇒ design ok
- Fuel elements: Idem Core Binder and:
  - 100 kWe OPUS version seems feasible ⇒ fuel fabrication?
  - 500 kWe OPUS version needs fuel design improvements



# Conversion Systems

## ① Brayton cycle

- He-Xe (85 g/mol)
- 3.6 kg/s
- $DT_{\text{core}} = 427 \text{ }^\circ\text{C}$
- $h = 27 \%$
- 50 m<sup>2</sup> of radiator
- 14 bars
- 900 kg



**100 kWe – 1300 K**

## ② Hirn cycle

- Sulfur, Alkali metal, ...
- 1 kg/s (S)
- $DT_{\text{core}} = 600 \text{ }^\circ\text{C}$
- $h > 50 \%$
- 7 m<sup>2</sup> of radiator
- 8 bars
- 620 kg

